



ON-GOING R&D, UPDATE AND SOME PERSPECTIVES ON CHROMIUM COATED Zr BASED CLADDINGS FOR ENHANCED ACCIDENT TOLERANT FUEL OF LWRs (CEA-EDF-AREVA FRENCH NUCLEAR INSTITUTE)

OECD-NEA 3rd meeting of the Expert Group on ATF of LWRs
Task Force 2 session

OECD-HQ – Issy les Moulineaux

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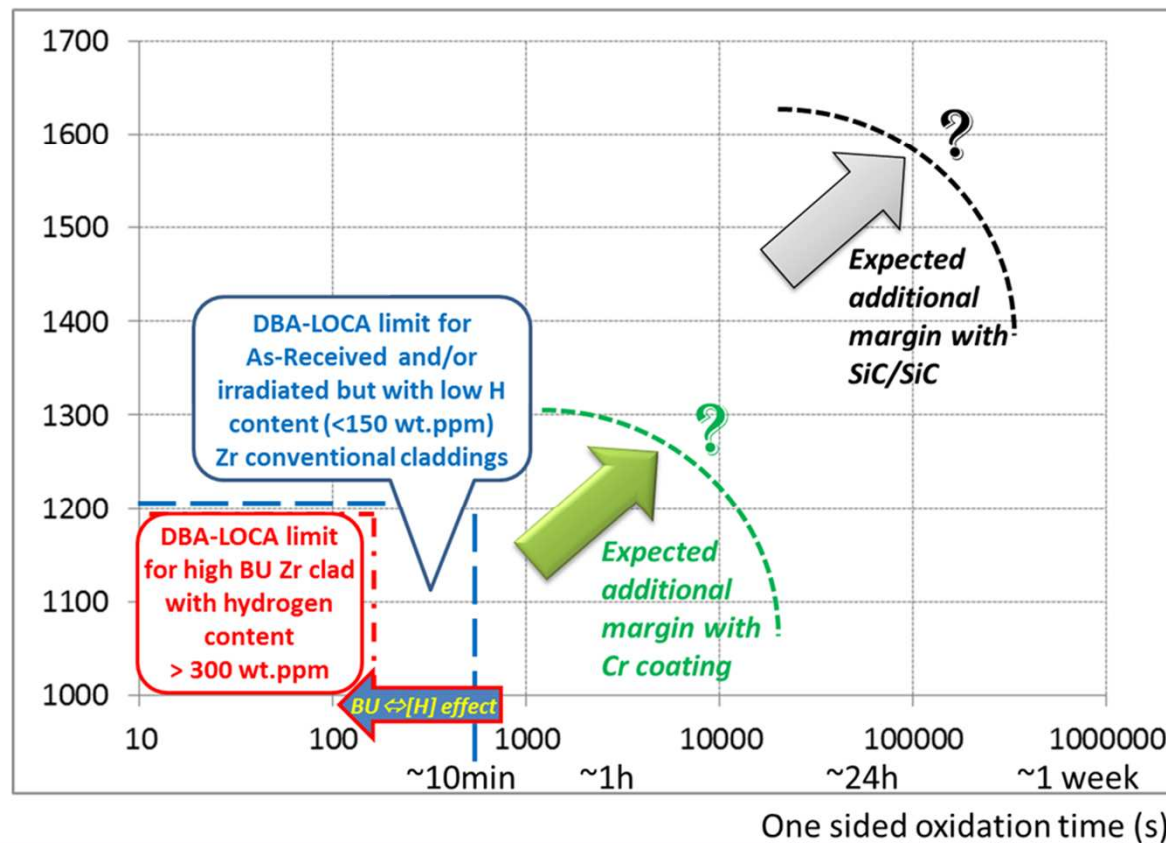
PROSPECTIVE STUDIES AT CEA ON ENHANCED ACCIDENT TOLERANT FUEL CLADDINGS FOR LWRS

Short/mid-term R&D:
1- Optimized Cr coating on Zr clad

Break-through/long-term R&D:
2- SiC_f/SiC + liner « sandwich-design » composite

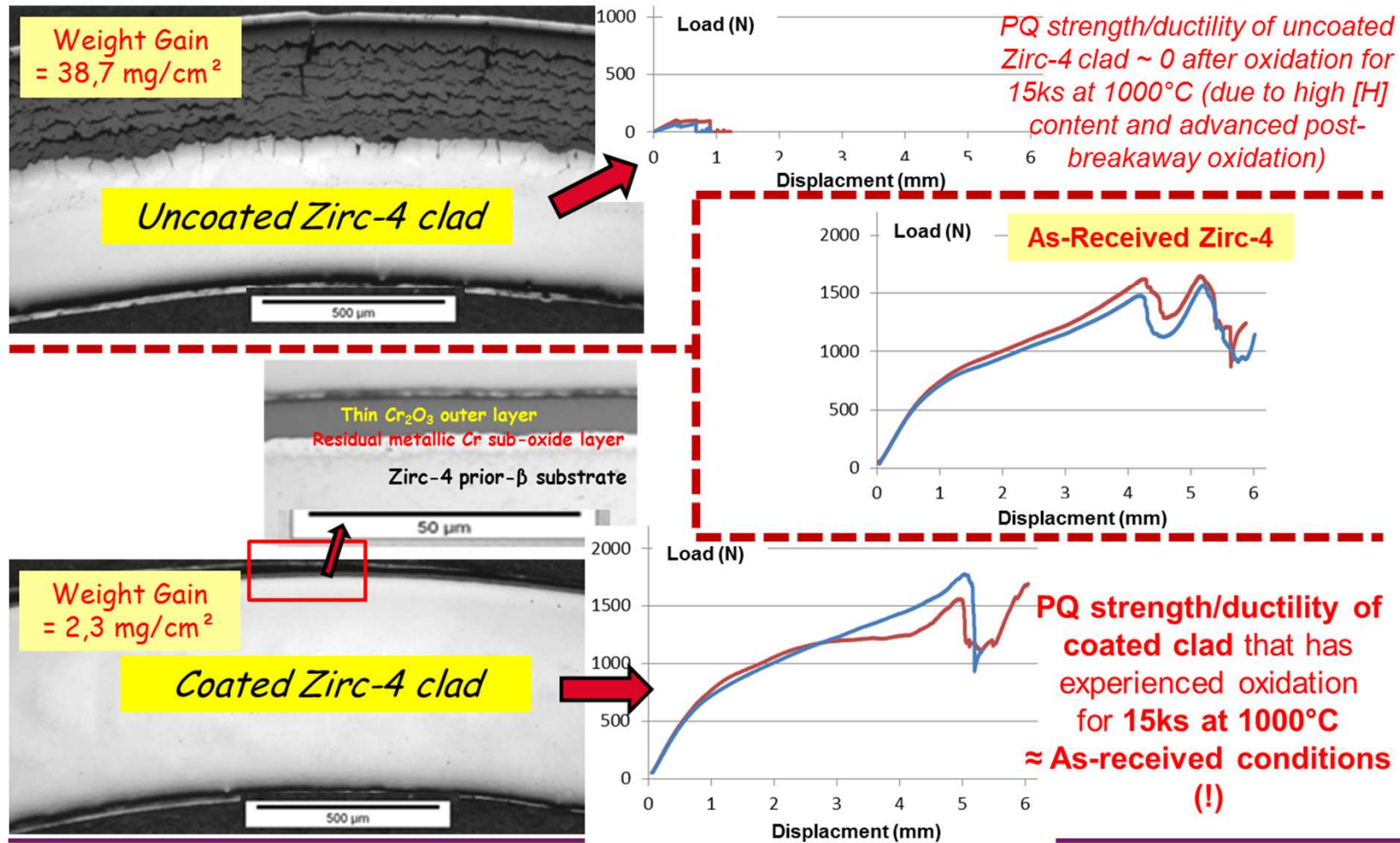
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Oxidation Temperature (°C)



■ **Beyond DBA-LOCA oxidation times:**
1000°C, >4h => "post-breakaway"
 (150mm length Zircaloy-4 based clad segments)

**Post-Quench (PQ) Ring
 Compression Tests (RCT)
 at 20-135°C**



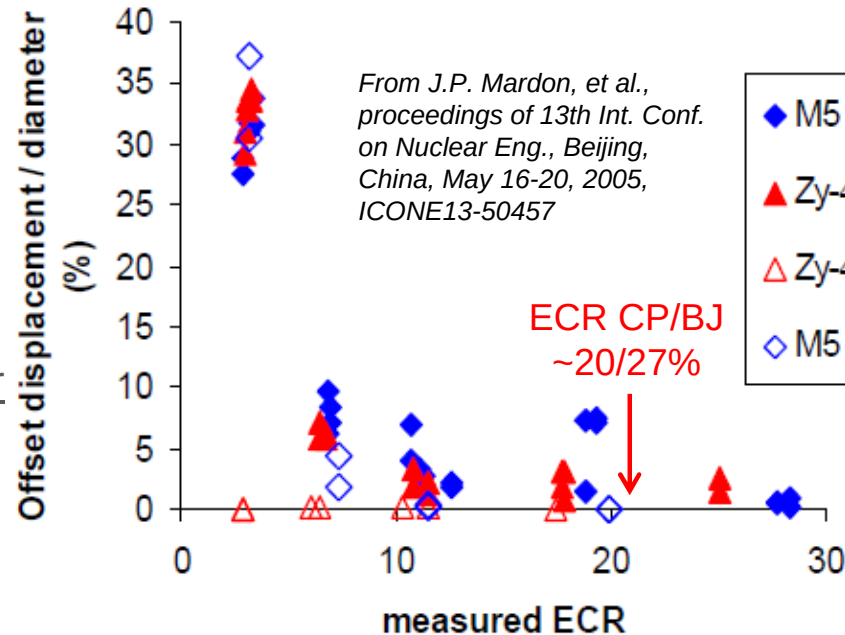
■ Slightly-beyond-DBA-LOCA-oxidation-time at 1200°C – 1500s :

(1) Results for uncoated Zirc-4 :

- One-sided oxidation
ECR CP/BJ ~20/27%

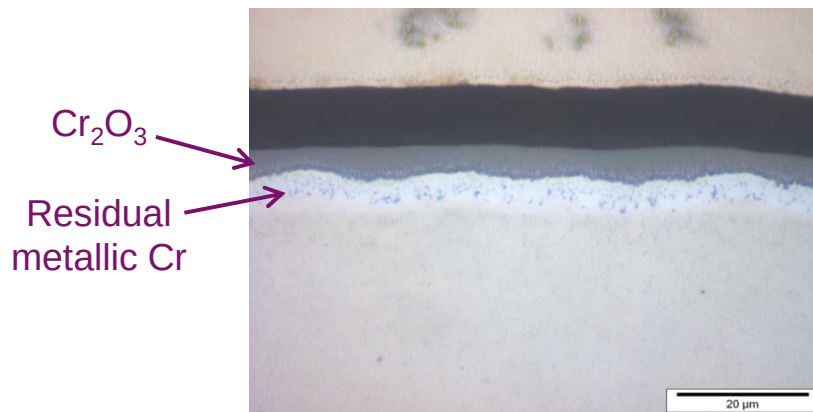
⇒ Typical phase thicknesses:
~115µm for ZrO_2
and ~140µm for the $\alpha(O)$ layer

⇒ Post-Quenching residual
ductility < 2%



(2) Results for 6-8µm Cr coated Zirc-4:

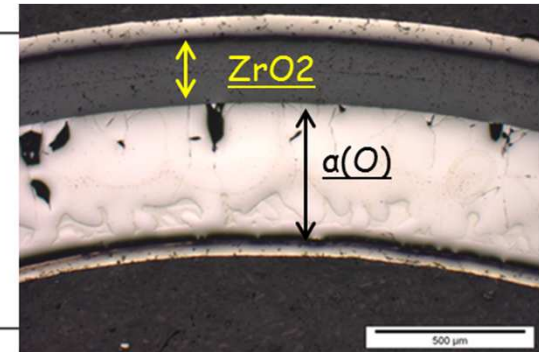
- Protective/sacrificial behavior of the coating
- ⇒ Negligible oxygen diffusion into Zr substrate
(no ZrO_2 , nor $\alpha(O)$ formation)
- ⇒ potential gain in the PQ residual ductility



■ **More beyond DBA-LOCA oxidation times:**
1200°C, >1h30 (150mm length Zircaloy-4 based clad segments)

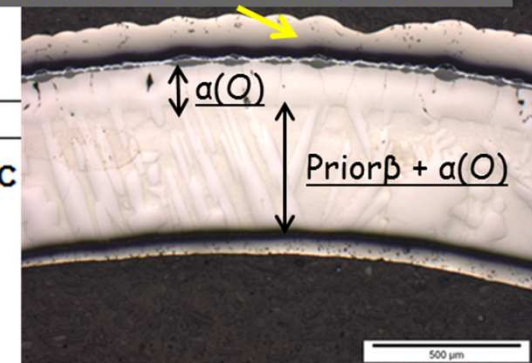


(a) Uncoated Zircaloy-4 clad segment, weight gain = 40,4 mg/cm²



(b) Coated Zircaloy-4 clad segment, weight gain = 11,4 mg/cm²

Sample preparation => outer Ni layer deposition

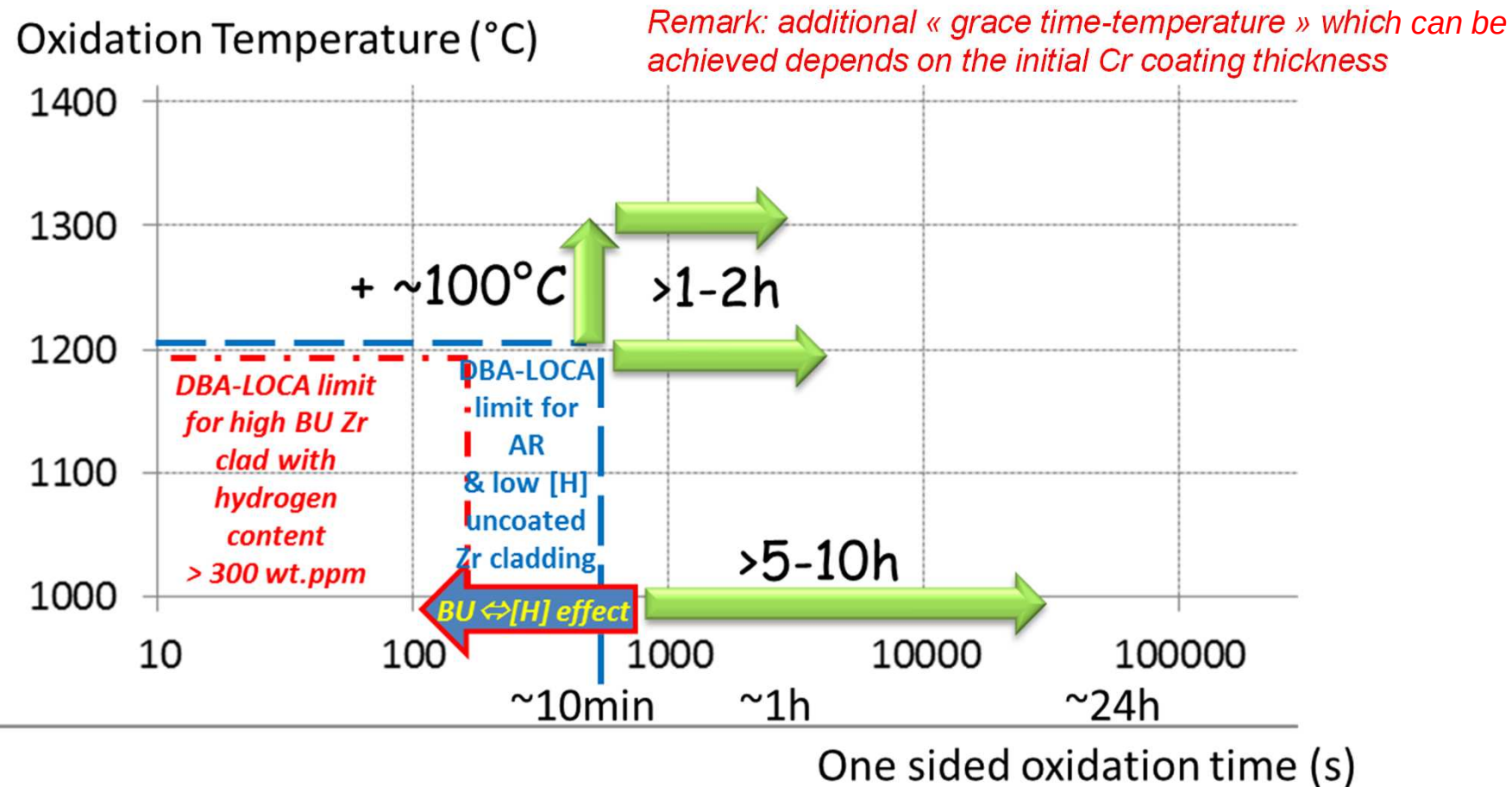


PQ Zircaloy-4 clad segments appearance after steam oxidation for 6000 s at 1200°C

Cr coatings improve the capacity of the HT oxidized cladding to maintain some residual ductility and to survive the final quenching for beyond-DBA-LOCA oxidation times at 1200°C, thus providing significant additional “grace period”:

=> 6000s is **ten times higher than the current DBA-LOCA maximum oxidation time at 1200°C** (~600s for as-received materials for one sided oxidation)

Expected additional HT steam oxydation time/temperature margins for fresh (~5-20µm thick) Cr coated Zirc-4 cladding vs. current DBA-LOCA limits





ON-GOING AND FURTHER WORK...

On-going

- On-going **autoclave testing** both at CEA and at AREVA (PWR pressurized water - 360°C, 100 bar – steam – 415°C)
- **Multi-scale metallurgical examinations** (XRD, SEM-EBSD, TEM observations of thin foils from FIB preparation...), micro/nano-hardness measurements...
- Additional **HT steam oxidation and post-quenching mechanical tests**
- **Transposition of deposition process to longer cladding tubes** (from 150 to 500 mm at first)
- **First irradiation performed on Cr coated Zr sheets specimen using charged particles** (Kr), thanks to CEA-Saclay “Jannus” triple-beams accelerator => on-going examinations (TEM, nano-hardness measurements...)

Mid/longer term Prospects

- **Ballonning/burst tests in LOCA conditions** (thanks to CEA “EDGAR” facility)
- **Other mechanical testing**: tensile, creep, internal pressure (CEA, AREVA...)
- **Longer corrosion tests** in nominal conditions (AREVA...)
- **Neutron irradiations** - Ex.: Halden, *CEA recommendations about fuel and cladding materials – including CEA Cr coated rods - to be included in the new experiments proposed for the 2015-2017 HRP, sent recently to HR board*